

Nuclear technology of Fuji Electric

We contribute to the realization of a safe, serene, sustainable society.

Undertakings in the Nuclear power business

3 unique technologies -- remote handling, waste treatment, and high temperature gas-cooled reactor--have ensured energy resources for more than half a century.*

Remote handling technology

- Nuclear fuel handling and storage equipment
- Remote transfer of high level waste
- Remote dismantling of nuclear reactor equipment
- MOX fuel fabrication equipment
- In-cell equipment

Fuji Electric nuclear power core technologies

Radioactive waste treatment technology

- Advanced solidification technology (geopolymer)
- · Spent resin volume reduction equipment
- Liquid waste equipment (fast breeder
- reactors, etc.)
- Remote cutting devices

Nuclear reactor engineering technology

- High temperature gas-cooled reactors (reactor internal structures, fuel handling equipment)
- · Criticality test equipment
- Nuclear fusion reactor related equipment

*Fuji Electric constructed Japan's first commercial nuclear power station, Tokai Power Station (a gas-cooled reactor), In 1965. It has also developed remote handling technology for radioactive materials, and superior technology for handing and processing radioactive waste, and had established unclear reactor technologies for high temperature gas-cooled reactors. Fuji Electric has played an important role in the development of the nuclear fuel cycle such as in the Fugen advanced thermal reactor, the Monju fast breeder reactor, high temperature gas-cooled reactor (HTTR), and fuel fabrication.

History of Major Achievements in the Field of Nuclear Power

Areas		1960's	1970's	1980's	1990's	2000's
Commercial	Tokai	Construction		In-vessel maintenance equipment	Confirmatory test for re	mote dismantling of the reactor
Reactor	Reactor					[Decommissioning]
Advanced	HTGR			High Tomporature	Engineering Test Reactor HTTR HTT	R spent fuel storage facility
Reactor	піан		Helium Engi	neering Demonstration loop HENDEL		In spent ruer storage raciity
					internals, Fuel handling system]	
			[Fuel stuck,	In core Structure test section]	Development	of demonstration reactor
	FBR		JOYO		JOYO Mk	
	FDN		JOTO	MONJU	JUYUWK	
		[Fuel	handling and storage]	IVIONJO	[Automatic control system for	fuel handling system]
			[Fuel	handling, Engineering Safety Features,	and etc.] Development of	demonstration reactor
	ATR		FUGEN			
			FUGEN			
		[Eucl bandling o		equipment]		
Nuclean Fuel ([Fuel handling e	quipment, Engineering safety protective	equipment]		
Nuclear Fuel (Cycle	[Fuel handling e		equipment]	MOX fuel fabrication facility	
Nuclear Fuel ((MOX fuel fab Reprocessing)	Cycle	[Fuel handling e			MOX fuel fabrication facility	facility]
(MOX fuel fab	Cycle	[Fuel handling e				
(MOX fuel fab	Cycle	[Fuel handling e			on facility, Pellet storage and transport Reprocessing plant	at Rokkasyo
(MOX fuel fab	Cycle	[Fuel handling e			n facility, Pellet storage and transport	at Rokkasyo
(MOX fuel fab Reprocessing) Radioactive w	Cycle rication	[Fuel handling e			n facility, Pellet storage and transport Reprocessing plant [Glove box equipment, li Renewal at JOYO	at Rokkasyo
(MOX fuel fab Reprocessing)	Cycle rication	[Fuel handling e	quipment, Engineering safety protective	[Pellet inspection of the second seco	n facility, Pellet storage and transport Reprocessing plant [Glove box equipment, li Renewal at JOYO IC plasm	at Rokkasyo
(MOX fuel fab Reprocessing) Radioactive w	Cycle rication	[Fuel handling e	quipment, Engineering safety protective	[Pellet inspection	n facility, Pellet storage and transport Reprocessing plant [Glove box equipment, li Renewal at JOYO IC plasm	at Rokkasyo



Client	I. Remote handling technology	II. Radioactive waste treatment technology	III. Nuclear reactor engineering technology
The Japan Atomic Power Company(Tokai)	(Decommissioning)		
Japan Atomic Energy Agency(Tokai)	(J-PARC)		
Japan Atomic Energy Agency(Oarai)	loyo, httr)	(JOYO)	(HTTR)
Japan Atomic Energy Agency(Thuruga)	(FUGEN, MONJU)	(FUGEN, MONJU)	
Japan Nuclear Fuel Ltd, (Rokkasyo)	 (Transport systems) 		

J-PARC: Japan Proton Accelerator Research Complex

Remote handling technology

Remote dismantling nuclear reactor equipment

Customer: The Japan Atomic Power Company

Operation start: July 1966

• Output: 166 MWe Operation end: March 1998



Tokai Power Station



Reactor pressure vessel, internals and primary cooling system (Drawing by 3D CAD for dismantling simulation)

The reactor dismantling test model features a faithfully simulated main reactor internal structures and reactor pressure vessel (19 m in diameter) constructed of the same material and in the same size.



We have obtained various data useful for the remote dismantling of an actual reactor by conducting demonstration tests of the technologies to remotely dismantle nuclear reactor vessels and other equipment that cannot be brought close to humans because of the high levels of radiation after long-term operation and these technologies include cutting the reactor pressure vessel, moving pieces of the vessel using robot arms and segregating the radioactive substances.



Gripper

Reactor pressure vessel model

Remoto handing technology

MOX Fuel Manufacturing Facility (MOX:<u>Mixed Oxide</u>)

Main Equipment Supplied by Fuji Electric

- Pellet Grinding and Inspection Equipment
 - · Pellet size and density inspection system
 - Density measuring Device
 - Equipment to Inspect the Grinded Pellet
 - Grinding machine
 - Pellet surface inspection equipment
- Inspection Machine of Fuel Assembly
- Automatic Storage Facility of Uranium

Features

- · Simplification of the system by the function integration
- High Speed Processing (100,000 pellets a day)
- Minimum hold up
- Improved maintainability
- Remote & Automation





Photograph of Glove Box (Courtesy of JAEA)



1. Grinding Machine

Major Spec.

- Method: Dry type center-less grinding method
- · Function: Grinding of the pellet periphery



2. Pellet size and density inspection system

Major Spec.

- Method: Size measurement by laser instrument Weight measurement by electromagnetic balance type
- Function: Quality check by measurement of size and weight of the pellet and computing density



Pellet surface inspection equipment

Major Spec.

- Method: 3 ITV camera imaging
- Spec.: Equipment check the surface of both end and the body of pellets

Bird's eye view of grinding inspection equipment

Radioactive waste treatment

Advanced solidification technology (Geopolymer) "SIAL[®]"

SIAL[®] *1 is the geopolymer, which is applied for encapsulation of radioactive waste. Geopolymer is a general term for amorphous condensation polymer formed by reaction of alkaline component and aluminosilica powder, and attracts attention in terms of the Cs encapsulation effect. It can be kneaded and solidified with a similar system to cement. SIAL[®] can also solidify waste, such as used radioactive ion exchange resin, sludge waste, incineration ash, oil and concentrated liquid watse etc.

Feature

- · High ability to encapsulate Cs and heavy metals
- Physically and chemically stable
- · Wide range of solidification target waste
- Easy handling (high liquidity, no heating required, on site treatment with mobile facility)
- High waste loading ratio (Reduce number of waste body)



SIAL® Solidified samples

Waste loading ratio	MAX 40 wt%
Compressive Strength	7-45 MPa
Density	1.4 to 1.8 g/cm ³
Viscosity	∼2Pa•s



Treatment facility

Solidification test of radioactive waste from NPP*2

- Conducted the first demonstration test of radioactive waste solidification by geopolymer in Japan
- Basic data acquisition of the solidified body for sludge and ion exchange resin produced by the Tsuruga Nuclear Power Plants
- The impact of Isosaccharinic Acid (ISA*3) on the partition coefficient is small

Solidification and Characterization



*1 SIAL is a licensed technology of Jacobs

*3 Isosaccharinic acid, decomposition product of cellulose, is concerned about the decline of Partition coefficient.

*4 No. 3 Waste Disposal Facility at Rokkasho Low-Level Radioactive Waste Disposal Center

^{*2} A part of the results of the contracted research program with Japan Atomic Power Company.

Nuclear reactor engineering

High temperature engineering test reactor (HTTR)

- Customer: Japan Atomic Energy Agency (JAEA)
- Output Power: 30 MWth
- Outlet coolant temperature: 850°C/950°C
- Main products: Reactor internal structures, Fuel handling & storage system
- Core design, Safety analysis: in cooperation with JAEA



Reactor internals (top view, outer dimension 4.25 m)

'HTTR' Fuel Handling Machine

• Feature

- Fuel handling machine is used for charging/discharging of fuel blocks into/out of nuclear reactor and is used for transfer of fuel blocks between spent fuel storage facilities and fresh fuel storage facilities
- Fuel remote and automatic operations perform a series of refueling, and position fuel assembly blocks to any predetermined places inside reactors.

Design pressure	98 kPa
Design temperature	200°C
Dimensions	11 m
Weight	150 t
Atmosphere (external / internal)	Air / helium gas
Handling load capacity	Fuel assembly block, etc. 200 kg





Fuel Handling Machine (Gripper)

For Fuji Electric Co., Ltd.

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